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# IER 551: EUROPA Intermediate Pu and Be Experiment

Peter Brain, C. Kostelac, N. Thompson  
A. McSpaden, and G. McKenzie

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# Overview

## 1. Introduction

1. Validation of Pu Cross Sections
2. Plutonium Benchmarks

## 2. Experiment for Unresolved Resonance Of Plutonium Actinides (EUROPA)

1. Creation of series of true intermediate benchmarks via Particle Swarm Optimization (PSO)

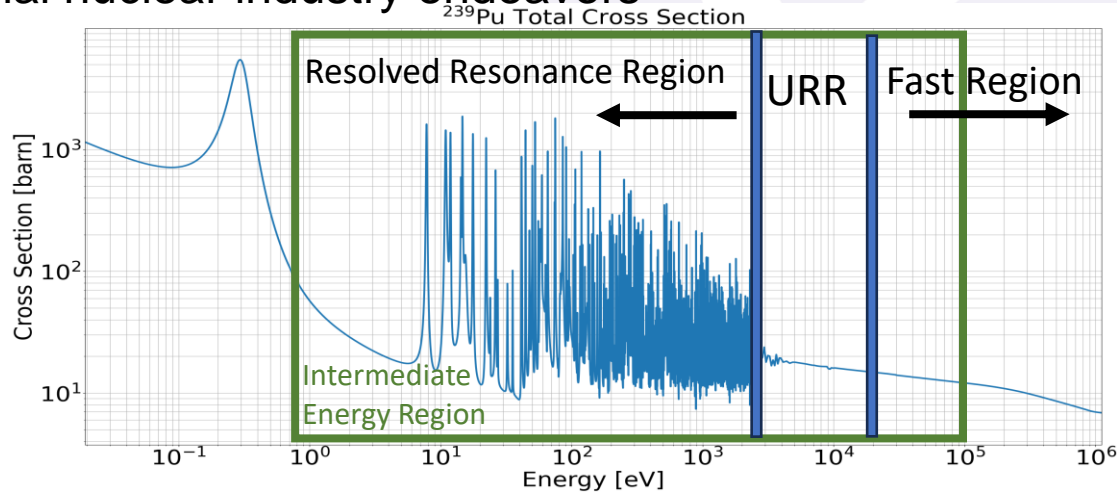
## 3. Summary

1. Preliminary Design and Secondary Considerations

# Nuclear Data

## Connecting the RRR, URR, and Fast Regions of Pu

- Intermediate Energy Region (0.65 eV to 100 keV) contains all three evaluation methods of Pu
- Due to limitations of theory and differential data, there's always a need to validate the cross sections using integral experiments
- Pu isotopes are critical to the national stockpile stewardship, non-proliferation, and commercial nuclear industry endeavors



# Summary of Current ICSBEP Benchmarks

- Currently around 48 cases of (near) intermediate configurations.
  - PU\_MET\_INTER(PMI) 02-04 are ZPPR assemblies at ANL (1980s)
  - PU\_MET\_MIXED(PMM)-01 are BFS critical assemblies (1960s)
  - PMM-02 is TEX-Pu done at NCERC (2017/8)
  - PU\_COMP\_MIXED(PCM)-02 are assemblies from Hanford crit mass lab (1960s)
  - PU\_COMP\_INTER(PCI)-01 is a k-infinity benchmark done at HECTOR in UK (1960s)

Benchmark	C/E $k_{eff} \pm 1\sigma$	Main Moderator(s)	Intermediate Fissions
PMI-02	$0.997 \pm 0.0023$	Graphite/Stainless Steel	66.38%
PMM-01	$1.01 \pm 0.0037$	Polyethylene	61.55%
PMM-02	$1.002 \pm 0.00260$	Polyethylene	42.76%
PCM-02	$1.04 \pm 0.0046$	Polystyrene	19.39%
PCI-01 (k infinity)	$1.001 \pm 0.0110$	Graphite/Boron	88.36%

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There are no modern Pu system that are truly intermediate and reliable enough for ND validation.

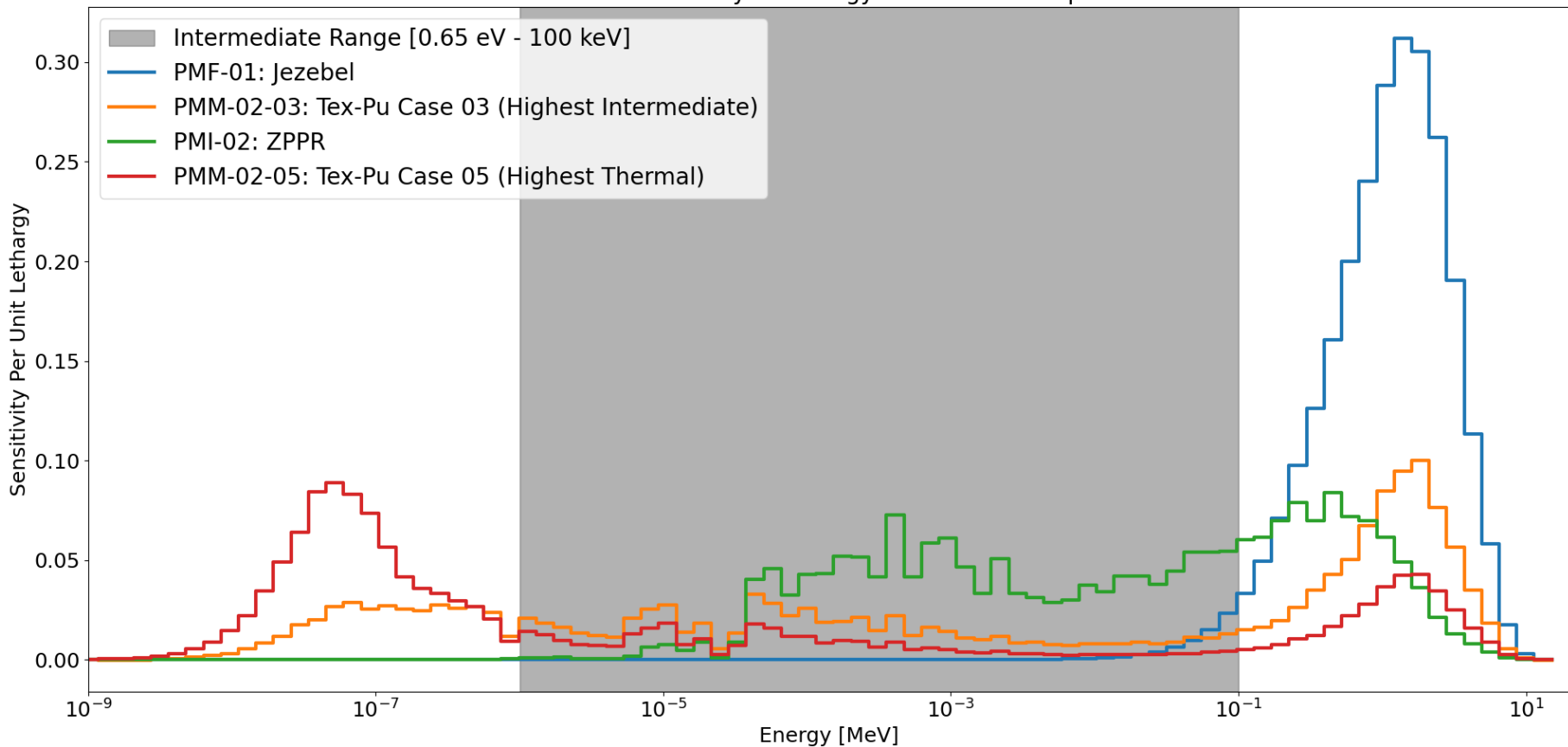
# Targets for Intermediate Pu Systems

## Cross Section Sensitivities for Pu Intermediate for 0.65 eV to 100 keV

- Besides PCI-01, all systems are below 0.19%/ % for fission sensitivity
- Similarly, capture sensitivity does not exceed -0.13%/ %

Benchmark	Capture Sensitivity (%/%)	Fission Sensitivity (%/%)
PMI-02	-1.3035E-01	1.9083E-01
PMM-01	-9.8473E-02	1.5946E-01
PMM-02	-8.5911E-02	1.4758E-01
PCM-02	-2.1148E-02	3.6065E-02
PCI-01 (k-infinity)	-1.6520E-01	5.2318E-01

$^{239}\text{Pu}$  Fission Sensitivity vs. Energy of Various Pu Experiments

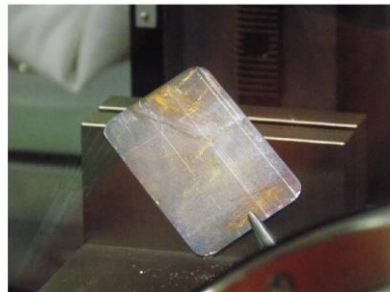




# Experiment for Unresolved Resonance Of Plutonium Actinides (EUROPA)

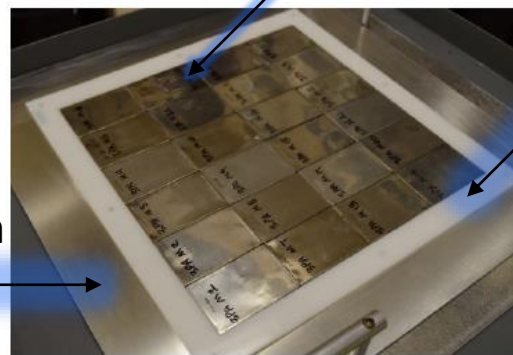
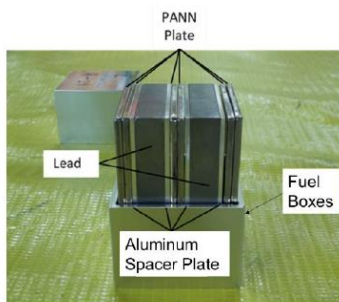
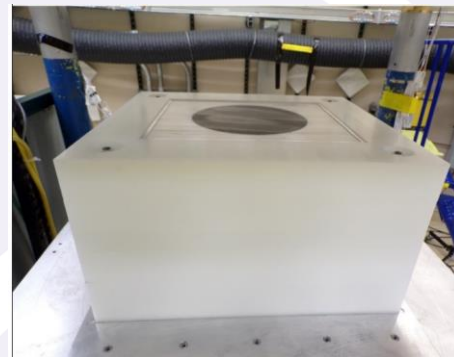
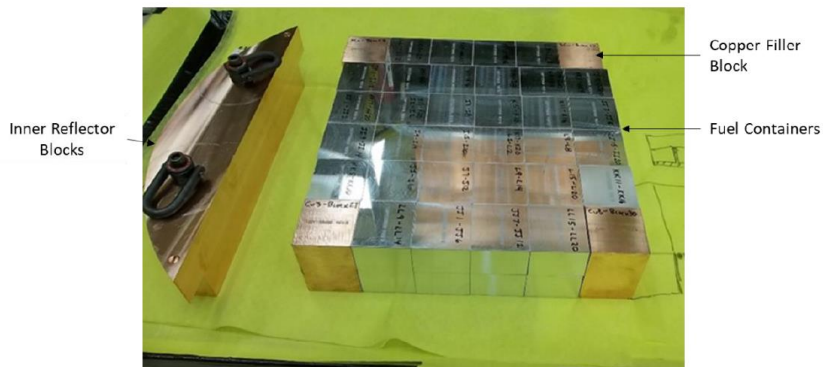
## **IER 551 True Intermediate Plutonium**

- Energy gap in available Pu benchmarks for intermediate energies
  - 0.65 eV – 100 keV includes resolved, unresolved, and fast regions
- **GOAL**
  - **1) Maximum % of fissions caused by intermediate neutrons**
  - **2) sensitivities to Pu-239 fission > 0.2 %/%**
  - **3) and experimental uncertainties below 300 pcm**
- Use ZPPR PANN and PAHN (high Pu-240 content) plates to develop a series of true intermediate benchmarks, to validate current and future Pu evaluations



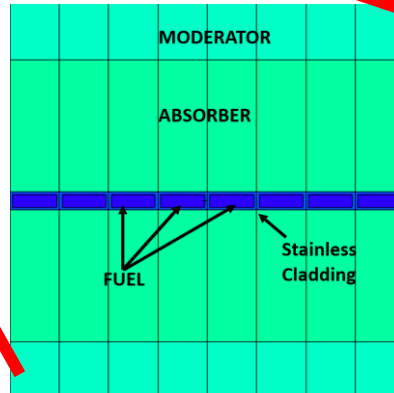
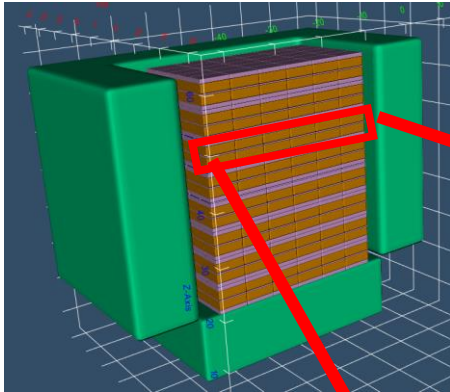
# Geometries, Materials, and Permutations

- Jupiter fuel block style vs. CWS/TEX layers
- Layer-style (CWS/TEX) designs win out due to ease of optimization



# Particle Swarm Optimization (PSO) for Material Selection

- An “optimized” design has high sensitivities and high percent of intermediate neutrons causing fission
- Need to perform an exhaustive material search to say that this is in fact optimized for intermediate

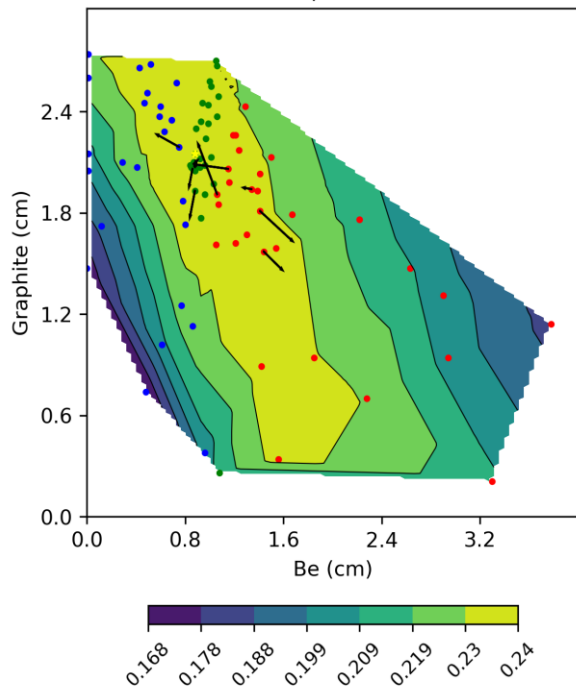


TYPE	Material	Nominal Density [ $\frac{g}{cm^3}$ ]
MOD / IR	HDPE	0.96
MOD / ABS	Borated <sup>a</sup> HDPE	1.04
MOD	Lucite	1.18
MOD / IR	Be	1.848
MOD	BeO	3.02
MOD / IR	Graphite	2.266
MOD / IR	Alumina	3.95
ABS	Cadmium	8.96
ABS	Gadolinium	7.90
IR	Stainless Steel - 304	7.5
IR	Copper	8.96
IR	Lead	11.37
IR	None (Air)	$1.25 \times 10^{-3}$

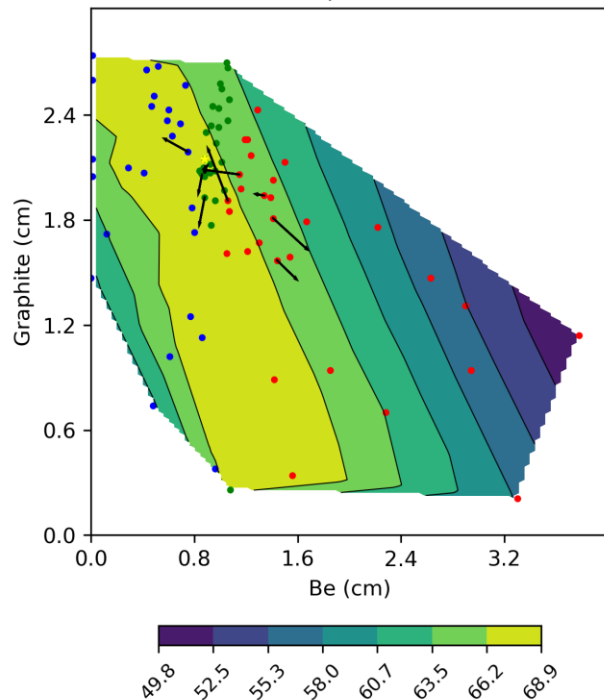
<sup>a</sup> Borated polyethylene is 5% boron by weight and enriched to 90% <sup>10</sup>B

# While PSO searches for intermediate fissions it also must be constrained to look for critical configurations

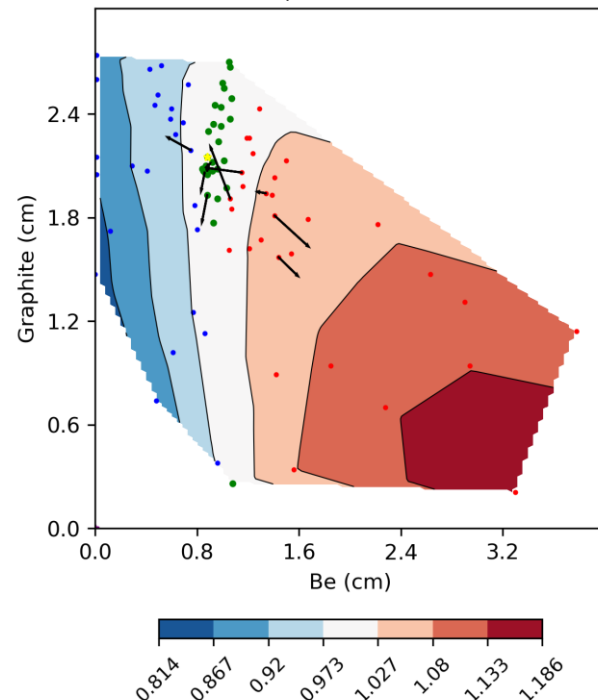
$S_{k_{eff}, \sigma}$  - Generation 9  
8 unit Europa Cu reflected



Epithermal % - Generation 9  
8 unit Europa Cu reflected



$k_{eff}$  - Generation 9  
8 unit Europa 15.0cm Cu reflected



# Results

## Beryllium material combinations take 8 out of 10 spots for highest intermediate neutrons with 8 fuel layers

- Look for additional high sensitivity and intermediate fissions as reactivity can be controlled orthogonally

Materials	Mod.	Abs.	$K_{eff}$	EALF	T %	I %	F %	Sens. $^{239}\text{Pu}$ (n,f)
BeO-Cd	3.19	0.01	0.99694	0.000509	6.35	73.03	20.62	0.31092
BeO-Gd	2.73	0.01	0.99134	0.000774	5.62	72.11	22.27	0.29581
Be-Cd	2.99	0.01	1.00095	0.000378	8	72.05	19.95	0.31683
Be-Gd	2.76	0.01	0.99011	0.00048	8.05	71.03	20.92	0.30839
Teflon-Graphite	3.64	1.29	0.73795	0.001753	3.83	70.73	25.44	0.25627
Graphite-Cd	3.35	0.01	0.86086	0.001648	2.96	69.89	27.16	0.27047
BeO-BHDPE	1.77	0.04	0.99869	0.001337	4.57	69.76	25.67	0.26556
Be-BHDPE	1.45	0.04	1.00335	0.001387	4.74	68.68	26.58	0.26985
Be-Cu	1.59	0.36	0.99351	0.001306	5.53	68.54	25.93	0.24239
BeO-Pb	2.26	0.63	1.0139	0.000711	8.8	68.06	23.15	0.23724
BeO-Graphite	1.25	2.02	0.99727	0.000518	10.52	67.33	22.14	0.24268
Graphite-BHDPE	3.29	0.08	0.80033	0.001154	5.98	67.21	26.81	0.29641
Teflon-Pb	3.17	0.78	0.67017	0.009289	0.4	67.12	32.47	0.18869
Graphite-Gd	3.97	0.06	0.68703	0.003171	1.7	67.02	31.28	0.29239
Be-Graphite	0.88	2.15	0.99387	0.000591	10.02	66.91	23.07	0.23932
Teflon-Gd	3.89	0.06	0.60972	0.011328	0.1	66.82	33.08	0.21661
Be-Pb	1.91	0.93	0.99803	0.000634	10.35	65.83	23.82	0.23387
Teflon-Cd	2.67	0.05	0.69425	0.014751	0.04	65.4	34.57	0.17028
BeO-Cu	3.12	0.48	0.99043	0.000287	14.77	65.08	20.15	0.25279
Graphite-Cu	2.79	1.38	0.69062	0.003158	3.28	64.86	31.86	0.24655
Cu-Graphite	0.07	2.07	0.85518	0.005552	1.33	64.52	34.16	0.19827
Alumina-Cd	3.64	0.06	0.6696	0.013079	0.07	63.26	36.67	0.18282
Cu-BHDPE	2.59	0.28	0.55907	0.004819	1.95	60.78	37.27	0.28903
Alumina-BHDPE	2.46	0.66	0.44649	0.003291	3.01	60.67	36.32	0.32683
Alumina-Pb	3.9	1.69	0.54587	0.015946	0.5	58.72	40.78	0.17359
Pb-BHDPE	3.55	0.4	0.50544	0.005099	2.41	58.02	39.57	0.2928
Teflon-BHDPE	3.76	1.07	0.32557	0.004612	2.93	57.01	40.06	0.31367
HDPE-Cd	0.59	0.01	0.79909	0.00036	15.47	56.94	27.59	0.28733
Lucite-Gd	1.91	0.06	0.43969	0.001016	9.35	56.73	33.92	0.33063
Lucite-Cd	0.98	0.01	0.75957	0.000251	18.09	55.76	26.15	0.29023
Graphite-Graphite	0.62	0.66	0.7955	0.026035	0.14	53.54	46.32	0.13174

# Preliminary Design Concepts

Achieved: Exceed 68% intermediate fissions and 0.2 Pu-239 fission sensitivity

How feasible are these designs?

- Is it better to lose a larger fission sensitivity for a better benchmark experiment?
- Is 0.1 mm of Cd a realistic material dimension to incorporate into a design?
- Maybe other moderator/interstitials will yield similar sensitivities and avoid both Be hazards and thin pieces
- Minimize number of reflector and moderating elements to highlight Pu isotopes

<sup>a</sup>Teflon-Graphite combination has been loaded with three additional fuel plates to bring predicted  $k_{eff}$  to critical.

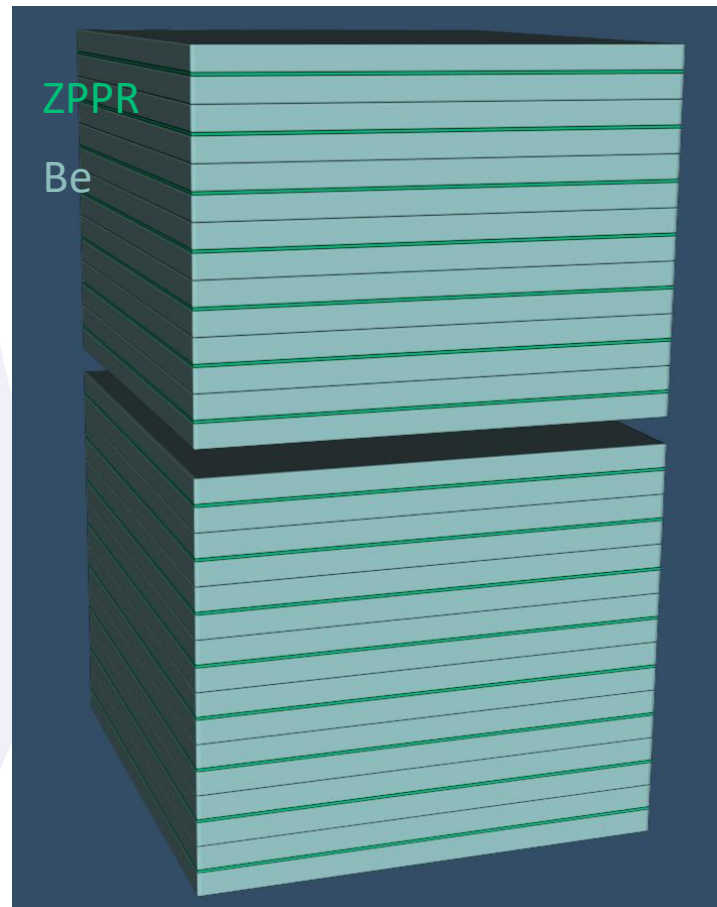
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Teflon_Graphite <sup>a</sup> :	3.64	1.29	1.00978	0.0043445	1.45	68.39	30.16	0.3190
HDPE_Pb:	0.32	0.45	1.01195	0.000229	25.22	46.92	27.87	0.16545
Be_Cu:	1.59	0.36	0.99351	0.001306	5.53	68.54	25.93	0.2423
Be_Cd:	2.99	0.01	1.00095	0.000378	8	72.05	19.95	0.31683
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# Final CED-01 Design of EUROPA

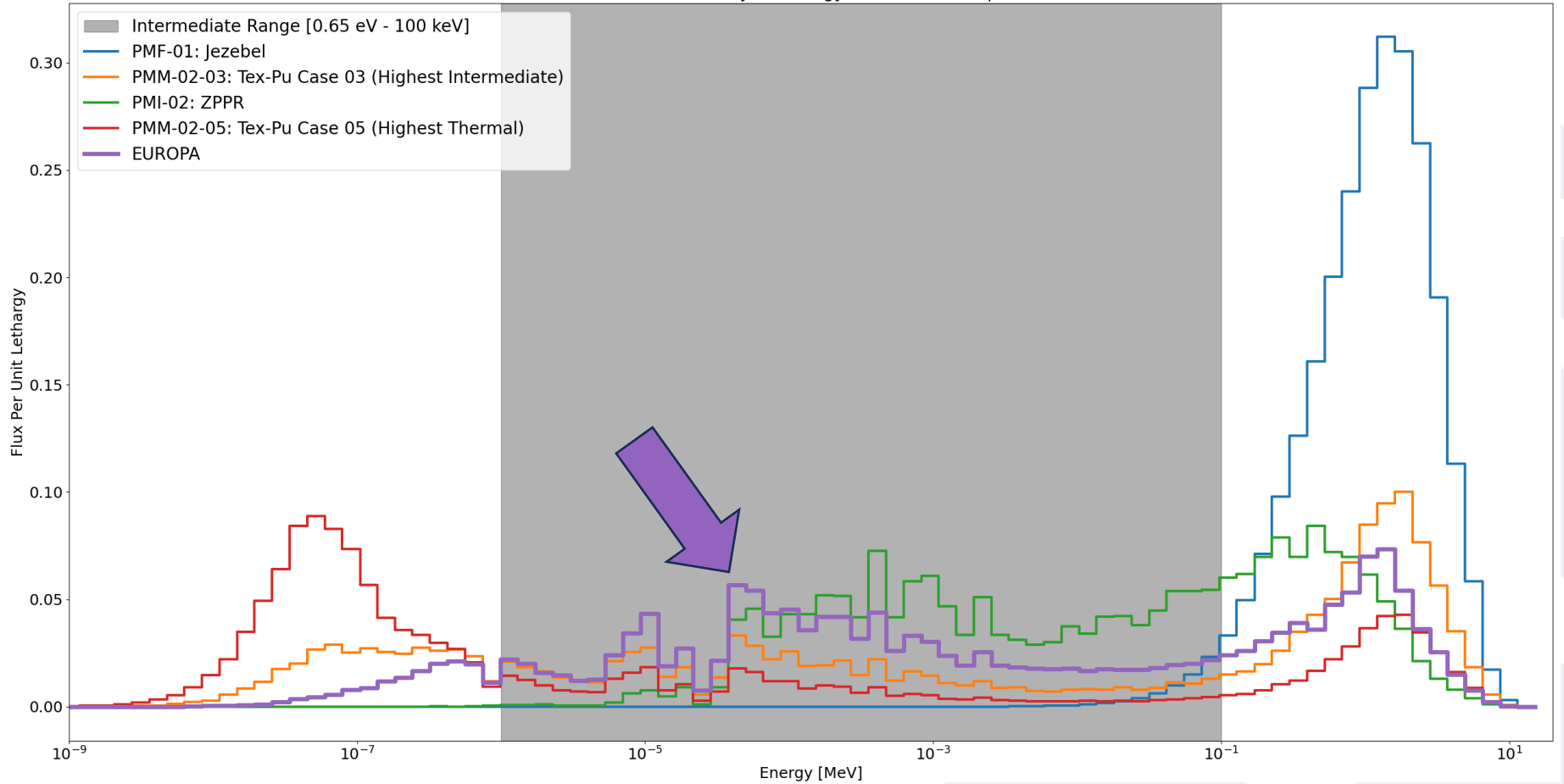
## Minimize materials other than Pu

- 16 - 7 x 5 fuel layers of ZPPR sandwiched between 1.8 cm of Be-metal
- More fuel layers and (n,2n) from Be allow for no reflector to be used

Model	Fission Sensitivity	Capture Sensitivity	Intermediate %
<b>EUROPA</b>	<b>0.3021</b>	<b>-0.1610</b>	<b>67.89</b>
PMI-02	0.1902	-0.1304	66.38
PMM-01	0.1594	-0.0980	61.55
PMM-02	0.1476	-0.0861	42.76
Jupiter	0.0127	-0.0054	16.07



$^{239}\text{Pu}$  Fission Sensitivity vs. Energy of Various Pu Experiments

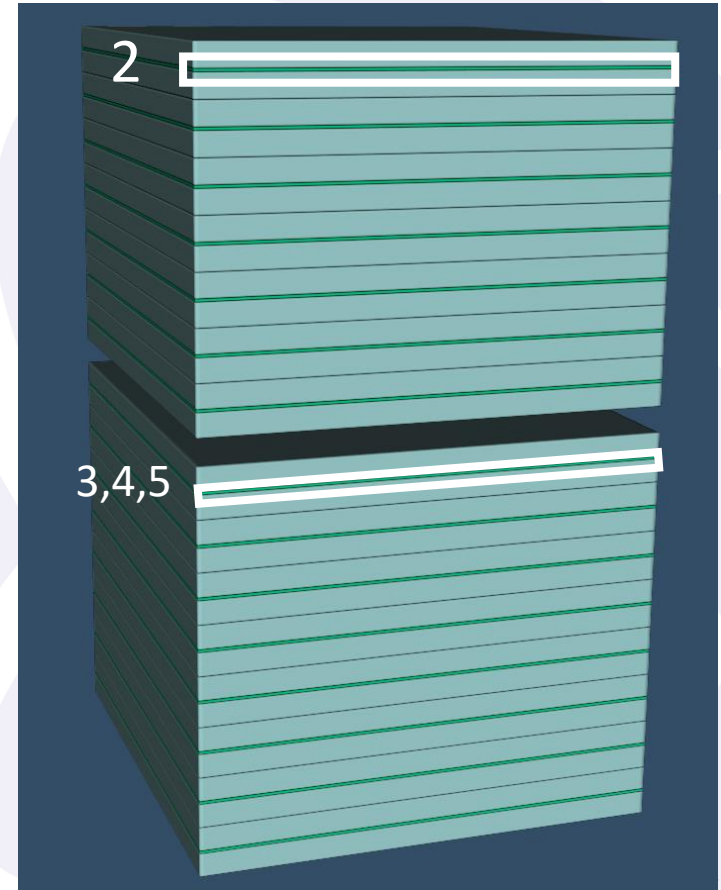




# Series of Critical Configurations

## Validate Pu-239 and Pu-240

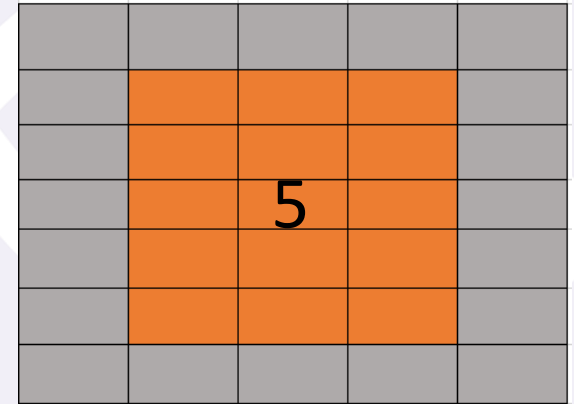
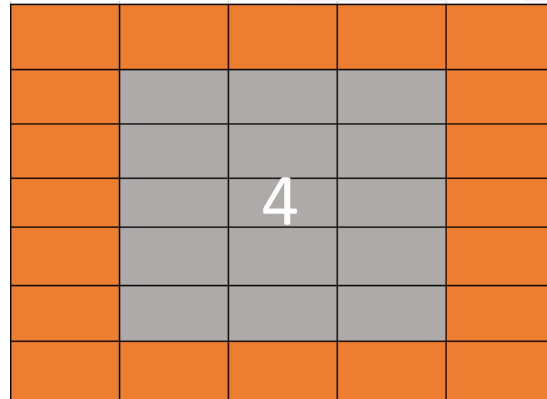
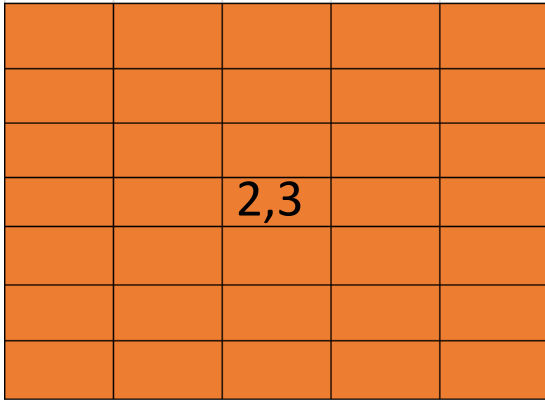
- PAHN plates will be used to add high Pu-240 content into system
- Pu-240 has higher capture cross section than Pu-239, needs to be varied throughout in the system
- 39 PAHN plates in inventory and each layer has 35 plates currently



# PAHN Experiment Layers

Need to introduce high Pu-240 plates into design to validate cross sections

- 1 is all PANN, nominal structure setup
- 2 is maximum PAHN on top of uppermost layer, 3 is top of bottom stack
- 4 is outer ring on top of bottom stack, 5 is center on top of bottom stack



# CED – 2 Discussions

## Looking ahead

- Is a thin graphite or aluminum reflector preferred to remove room return?
- If beryllium isn't available in the quantities, can Alumina/Graphite with Cu-reflector prove to be a suitable alternative?
- Sensitivities to Pu-240 calculations and final PAHN experiments

# Summary

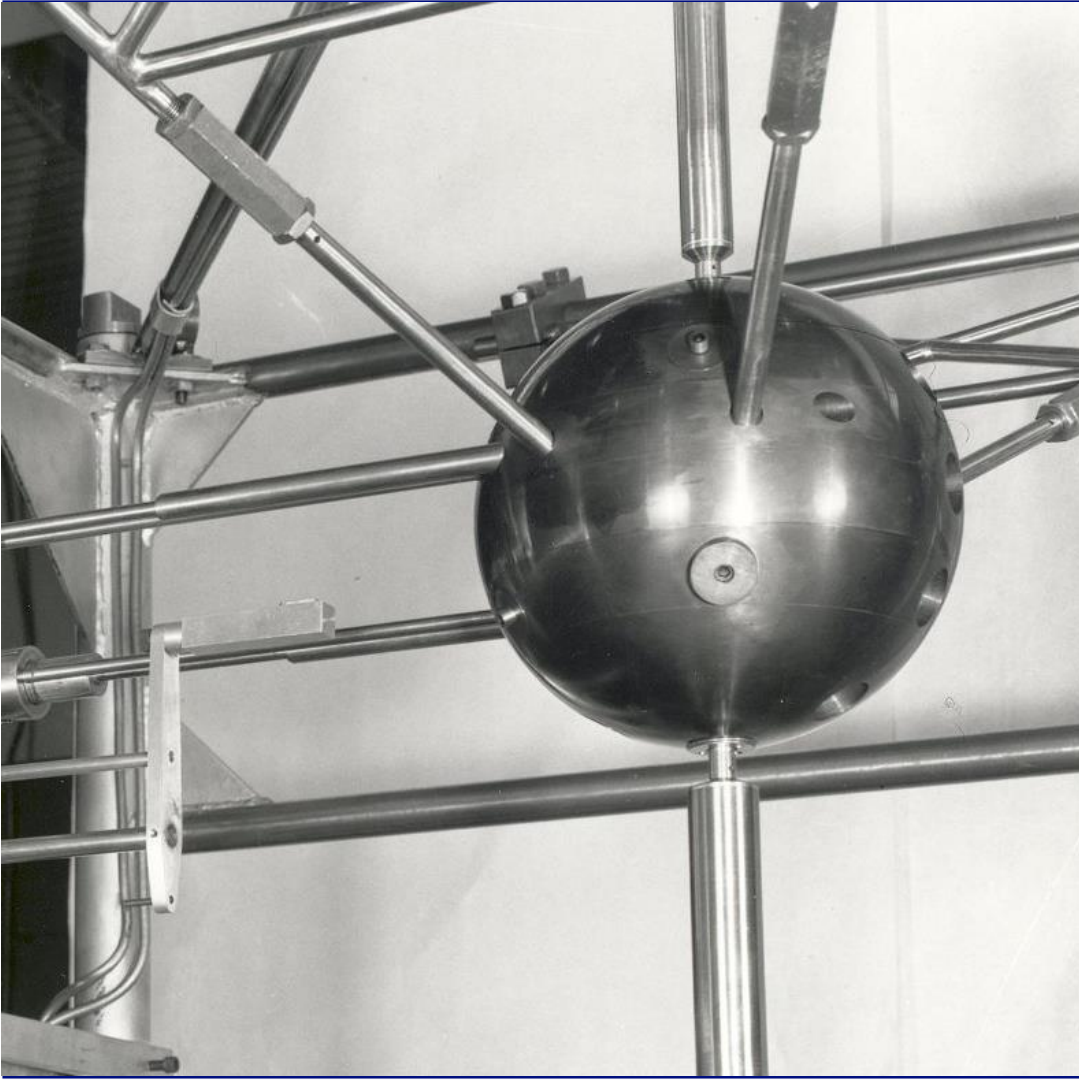
## Experiment for Unresolved Resonance Of Plutonium Actinides

- Motivation of validating end of RRR, whole URR, and beginning of fast region of Pu isotopes with new targeted integral measurement.
- Experiment design using ZPPR plates layered between moderators and absorbers optimized using PSO.
- Be-Cd interstitial combination proved to yield highest sensitivity to fission/capture cross sections with 8 fuel layers.
- Sole Be metal and ZPPR plate design chosen to move forward as it achieves design criteria with highest ND constraints

# Acknowledgements

- This work was supported by the DOE Nuclear Criticality Safety Program, funded and managed by the National Nuclear Security Administration for the Department of Energy.





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# Particle Swarm Optimization

## Gradient free optimization for Monte Carlo simulations by C. Kostelac

- Particle swarm optimization built on the premise of a swarm of particles traveling through a design space, position represented as design parameters

$$\vec{x}_i^k = \begin{bmatrix} x_1 \\ x_2 \\ \dots \\ x_n \end{bmatrix}$$

- Each particle updates its position in time with weighted knowledge of swarm best ( $\phi$ ), personal best ( $\rho$ ), and current velocity ( $\omega$ )

$$\vec{v}_i^{k+1} = \omega \vec{v}_i^k + \phi r_1 (\vec{p}_i^k - \vec{x}_i^k) + \rho r_2 (\vec{g}^k - \vec{x}_i^k)$$

- Swarm set to optimize for high intermediate fissions and fission sensitivities