

# NEA Nuclear Science Committee activities in connection with the NCSP

**Julie-Fiona Martin (NEA)**



- 1. The Nuclear Science Committee of the NEA**
- 2. WPNCS** Working Party on Nuclear Criticality Safety
- 3. WPEC** ... on nuclear data Evaluation Cooperation
- 4. WPRS** ... on scientific issues and uncertainty analysis of Reactor Systems
- 5. NEA tools**

# 1. The Nuclear Science Committee of the NEA

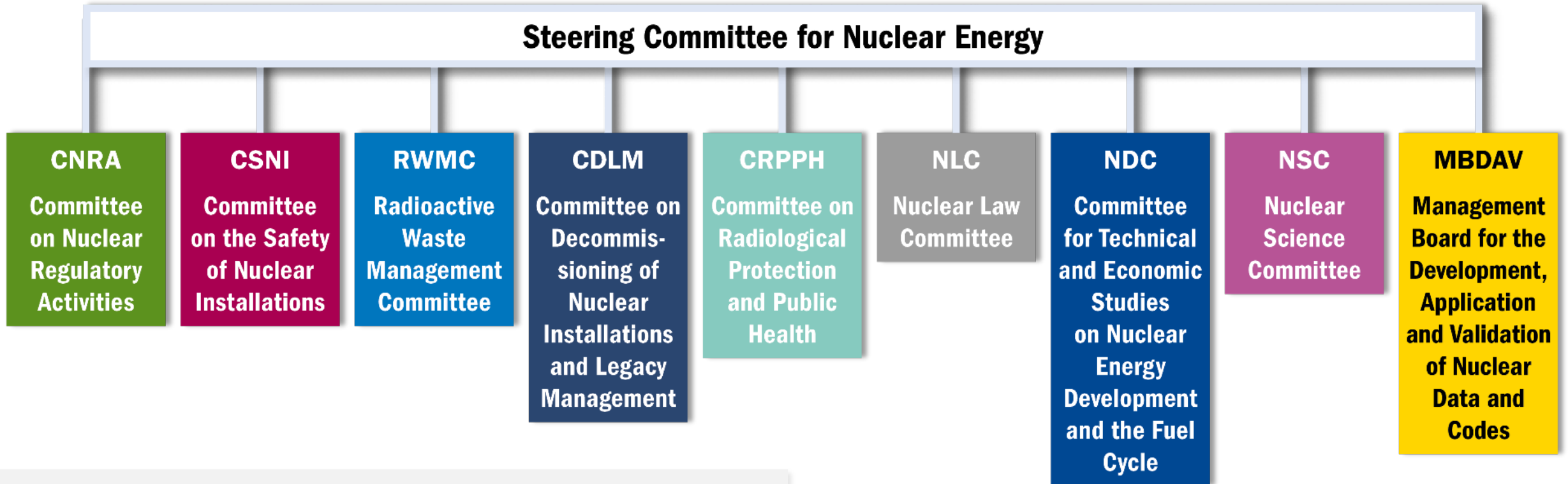
2. **WPNCS** Working Party on Nuclear Criticality Safety

3. **WPEC** ... on nuclear data Evaluation Cooperation

4. **WPRS** ... on scientific issues and uncertainty analysis of Reactor Systems

5. **NEA tools**

# NEA committees (as of 1 January 2022)



**8** standing technical committees  
**1** management board  
**74** working parties and expert groups

# Nuclear Science Committee (NSC)

## Working Party on International Nuclear Data Evaluation Co-operation (WPEC)

- Expert Group on the High Priority Request List for Nuclear Data (EGHPRL)
- Expert Group on the Recommended Definition of a General Nuclear Database Structure (EGNDS)

## Task Force on Demonstration of Fuel Cycle Closure including Partitioning and Transmutation (P&T) for Industrial Readiness by 2050 (TF-FCPT)

## Programme Review Group (PRG)

## Working Party on Scientific Issues of the Fuel Cycle (WPFC)

- Expert Group on Reactor Coolants/Components Technology (EGCoCoT)
- Expert Group on Fuel Recycling and Waste Technology (EGFRW)
- Expert Group on Advanced Fuel Cycle Scenarios (EGAFCS)
- Expert Group on Innovative Fuel Elements (EGIFE)

## Working Party on Materials Science Issues in Nuclear Fuels and Structural Materials (WPFM)

- Expert Group on Fuel Materials (EGFM)
- Expert Group on Structural Materials (EGSM)

## Working Party on Nuclear Criticality Safety (WPNCS)

- Technical Review Group for the International Criticality Safety Benchmarks Evaluation Project (ICSBEPTRG)
- Technical Review Group for the International Assay Data of Spent Nuclear Fuel Database (SFCOMPOTRG)

## Working Party on Scientific Issues and Uncertainty Analysis of Reactor Systems (WPRS)

- International Reactor Physics Experiments Evaluation Project Technical Review Group (IRPhETRG)
- Expert Group on Reactor Fuel Performance (EGRFP)
- Expert Group on Reactor Core Thermal-hydraulics and Mechanics (EGTHM)
- Expert Group on Reactor Systems Multi-Physics (EGMUP)
- Expert Group on Physics of Reactor Systems (EGPRS)

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# Working Party on Nuclear Criticality Safety (WPNCS) – Highlights since last NCSP presentation (Feb. 2022)



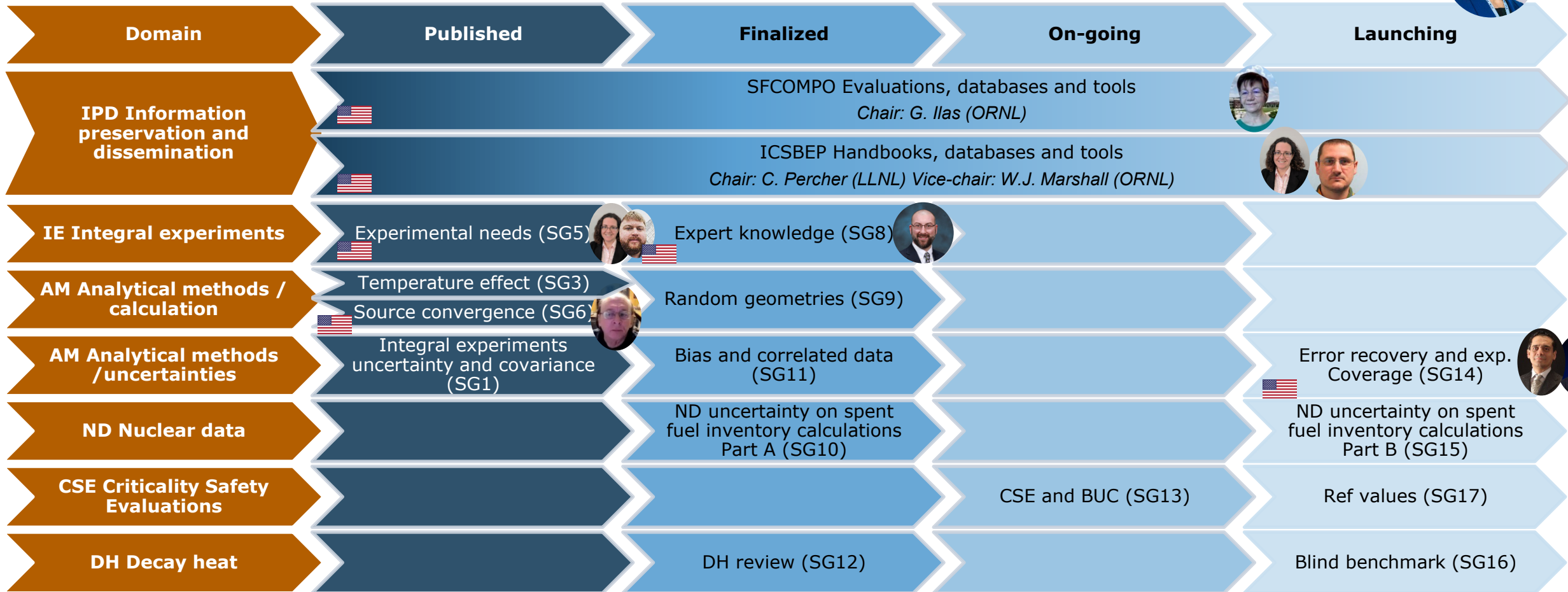
Chair: A. Vasiliev (CH)

Vice-chair: G. O'Connor (UK)

Secretariat: J.-F. Martin



US leadership highlighted



# Data preservation and dissemination

## SFCOMPO TRG Technical Review Group for the International Assay Data of Spent Nuclear Fuel Database

Chair G. Ilas (ORNL)

Nuclide inventory of used fuels experiments and decay heat measurements

24k measurements on 750 samples from 44 reactors, DH addition in progress

### Focus of the TRG

**Collection** of experimental data incl. decay heat data

**Evaluation** of the experimental data and the development of benchmarks and benchmark models

Revision of **guidance** documents

**Maintenance** of the database & GUI



[oe.cd/nea-sfcompo](https://www.oecd-nea.org/oe.cd/nea-sfcompo)

## ICSBEP TRG Technical Review Group for the International Criticality Safety Benchmarks Evaluation Project

Chair: C. Percher (LLNL); Vice Chair: B.J. Marshall (ORNL)

Critical, subcritical, alarm/shielding and fundamental physics experiments

587 experimental series from 19 countries, Handbook of ~70000 pages

### 2022 Edition

Annual meeting in Oct. + Dec '21 (remote)  
**6 new** evaluations approved + 1 update

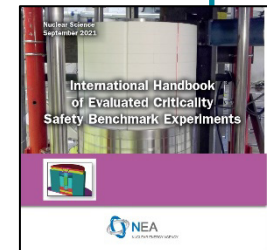
### 2023 Edition

Meeting in April 2023 (NEA HQ)  
**7 new** evaluations approved + 1 update

>> Joint 2022/2023 publication scheduled in '24

### 2024 Edition

Meeting in April 2024 (LLNL)  
**5 candidate new evaluations**



[oe.cd/nea-icsbep](https://www.oecd-nea.org/oe.cd/nea-icsbep)



# IE Integral experiments

## Experimental needs (SG5)

- Coord. I. Duhamel (FR), then C. Percher (LLNL) & G. McKenzie (LANL)
- Report published in 2022
- Review of needs, proprietary experiments, experimental facilities
- Needs ranked by usage / level of knowledge



1

Intermediate:  $^{240}\text{Pu}$  and  $^{238}\text{U}$ , Chlorine, Criticality safety training

2

Structural materials: Fe, Intermediate:  $^{239}\text{Pu}$  and  $^{235}\text{U}$ , Molybdenum, TSL: UZrH, Polyethylene at low temp, Solution reactor, Criticality studies and neutron source

3

Structural materials: Ta, Ni, Cr, Mn, Ni, F. TSL: HF, Lucite. Low and high temperature

4

Slab fuels, Structural materials: Si, W, Nb, Al

5

Structural materials: Zr

## Expert knowledge (SG8)

- Coord. W. Wieselquist (ORNL)
- Brief conclusions in preparation
- Acknowledge variation in quality expectations of ICSBEP benchmarks with time
- Capture tacit knowledge on quality/reliability/suitability of ICSBEP cases
- Template to be used for robust and long-lasting repository of feedback



# AM Analytical methods - calculations

## Temperature effect (SG3)

- Coord. S. Gan, A. R. Wilson (UK)
- Report published in 2023
- Comparison exercise assessing impact of temperature on  $k_{eff}$ , notably integrating TSL of H in water & ice, at various BU
- Experiment to confirm findings needed!



## Neutron source convergence (SG6)

- Coord. F. Brown (LANL, UNM)
- Report published in 2023
- Recommended series of statistical tests to enhance reliability and robustness of source convergence assessments in MC calculations
- Checked against a series of problems

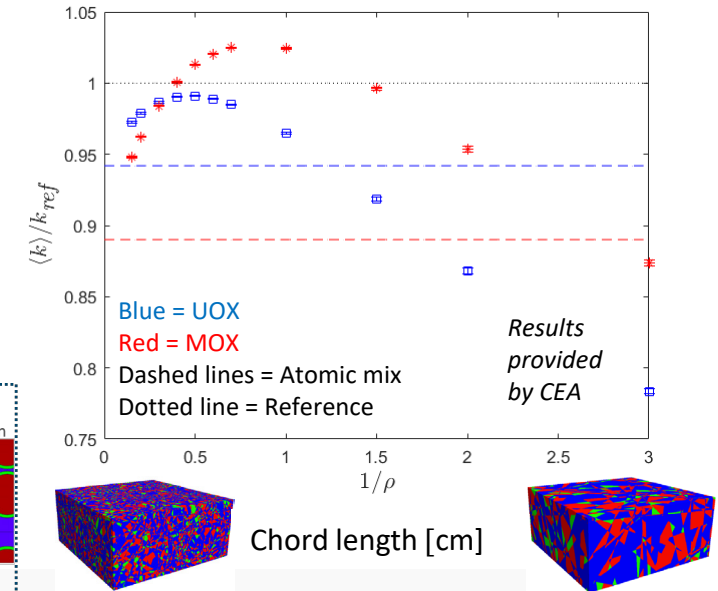
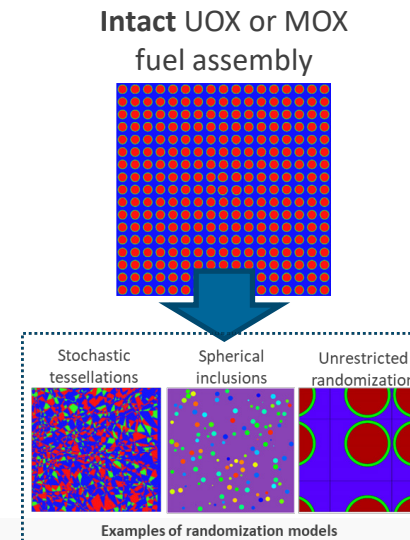


## Random geometries (SG9)

- Coord. A. Zoia (FR)
- Report pending publication
- Application cases : randomly arranged fuels (e.g. TRISO), severe accident recovery, dismantling, radwaste repositories
- Impact of randomization model on multiplication factor calculations



Left: Initial geometry, and randomized geometries. Right: Impact on randomization chord length on  $k_{eff}$   
 Court. A. Zoia, CEA (FR), Nov. 2022



# AM Analytical methods - uncertainties

## Integral experiments uncertainty and covariance (SG1)

- Coord. M. Stuke (GER)
- Report published in 2023
- Derive covariance between benchmarks and assess impact on application case



## Bias and correlated data (SG11)

- Coord. A. Hoefler (GER)
- Report in preparation
- Using pre-defined correlations between criticality benchmark experiments, assess role of methodology to derive impact on application case
- Families of methods provide consistent results



## Error recovery and experimental coverage (SG14)

- Coord. U. Mertyurek (ORNL) & H. Abdel-Khalik (Purdue U.)
- Kick-off in Sept. 2024
- Explore modern data assimilation techniques and capabilities to spot (and correct for) embedded errors and to quantify experimental coverage

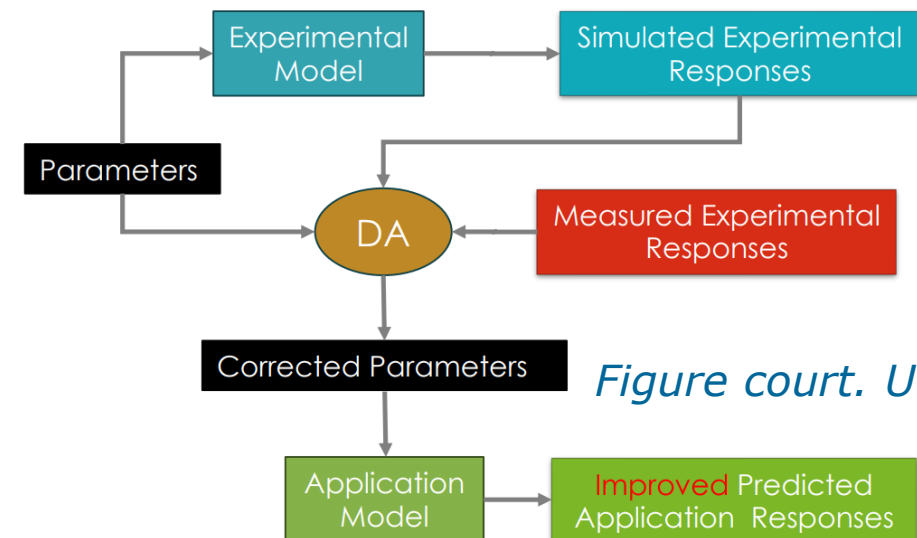
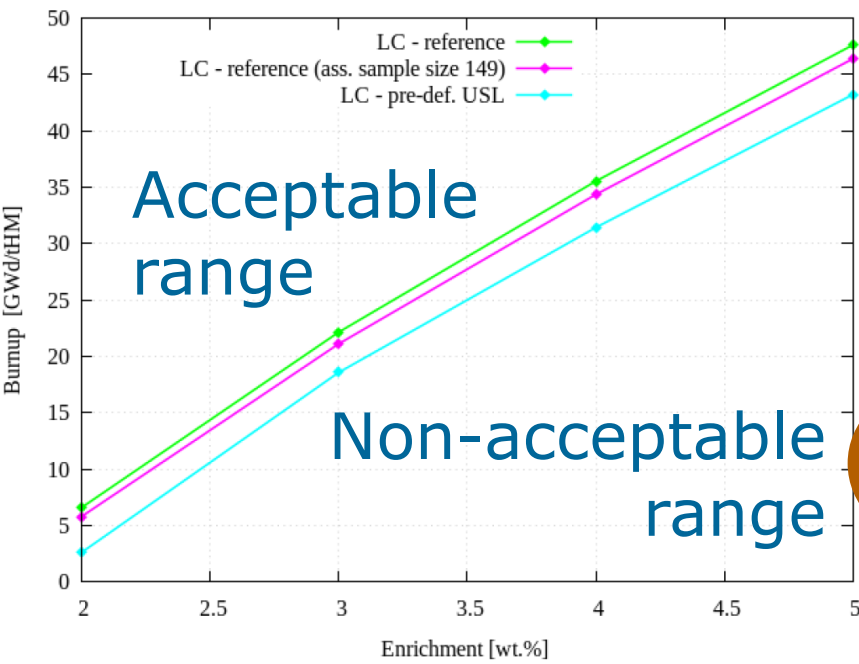


Figure court. U. Mertyurek

# CSE Criticality Safety Evaluations



Results court. PSI (CH)  
 LC = loading curve  
 USL = Upper Subcritical Limit

Blue = pre-defined USL  
 Green = with user-defined USL  
 Pink = with user-defined USL, relying on a larger validation base

## CSE + BUC, pseudo application case with user defined NCS criteria (SG13)



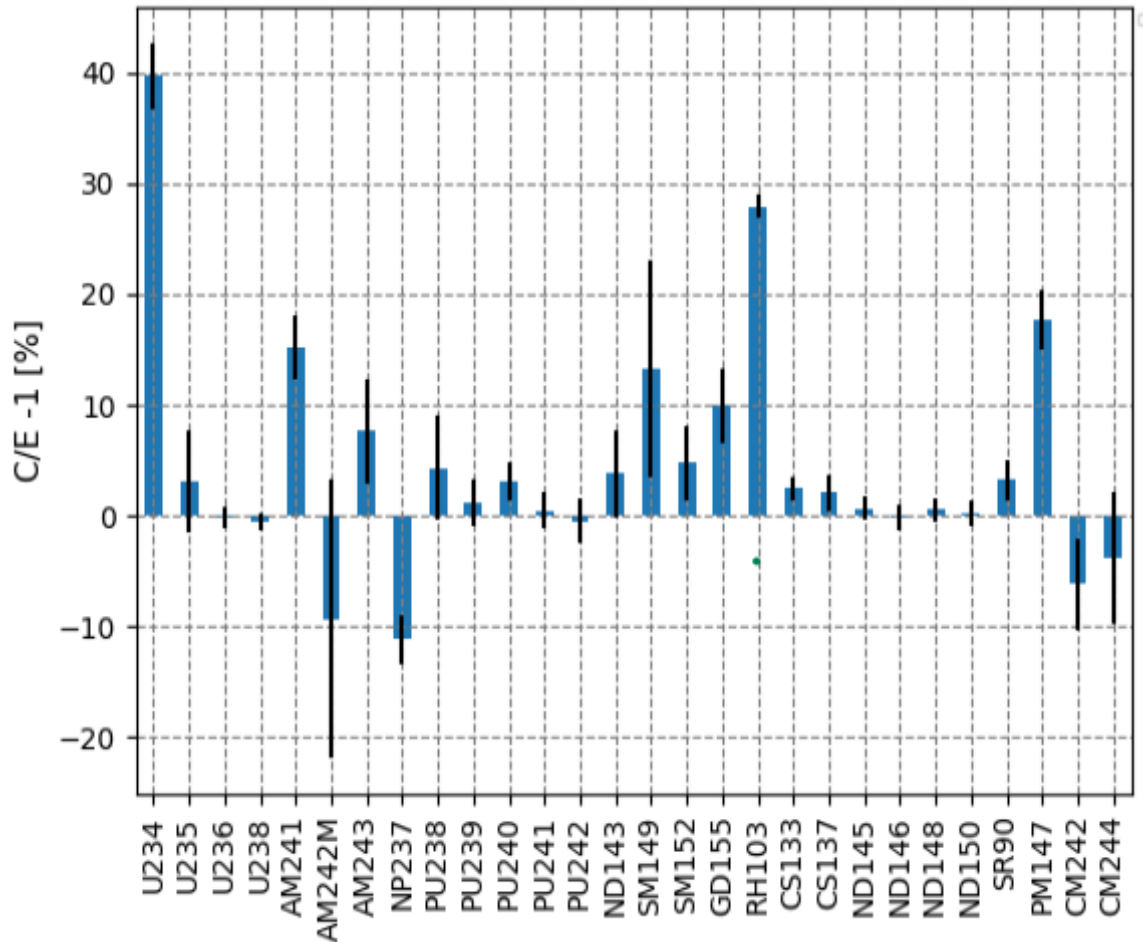
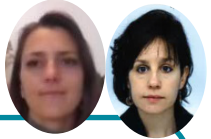
- Coord. A. Vasiliev & M. Wittel (CH)
- Started in 2023
- Simplified final disposal canister for used nuclear fuel
- Derive loading curve taking into account burn up credit with a user-defined nuclear criticality safety criteria (criticality aspects only – other metrics such as decay heat, radioprotection are not covered)
- Compare loading curves for various participants, covering variety of user-defined criticality safety criteria and methodologies

## Reference values for criticality (SG17)



- Coord. G. Frontier (FR)
- Kick off scheduled Sept 2024
- Provide reference values for Nuclear Criticality Safety (NCS) using state-of-the-art codes, methods and nuclear data

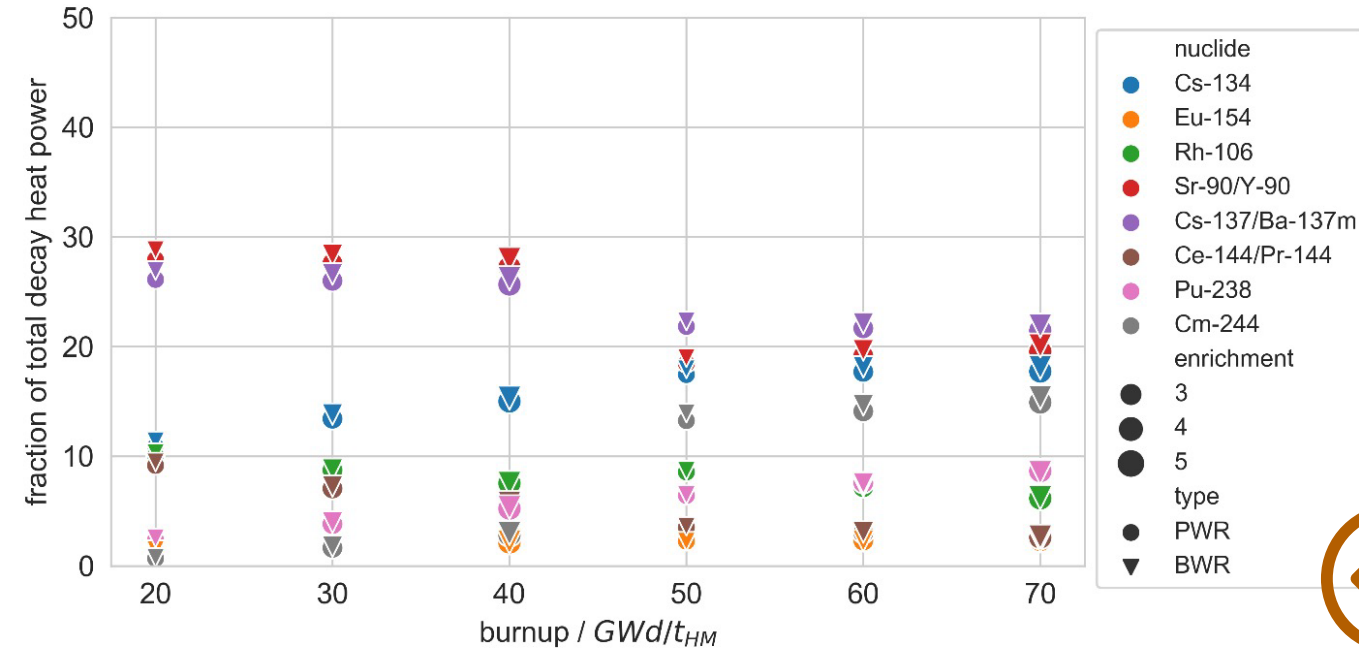
# ND Nuclear data



## Nuclear data uncertainty for irradiated fuels inventory calculations (SG10 & 15)

- Coord. C. Carmouze & R. Ichou (FR)
- Part 1: Best estimate values (2020-2023)
- Part 2: Uncertainty propagation (2023-2026)
- ARIANE GU3 spent fuel nuclide inventory
- Nuclear data sensitivity/uncertainty analysis (cross-sections, decay data and fission yield) on fuel assembly depletion calculations
- ~35 nuclides important for criticality safety and decay heat

# DH Decay heat



Main nuclides contributing to DH for UOX fuels at 5 years cooling time. From SG12 paper, under review. Coord. D. Rochman (CH)

## Assessing the confidence level in experimental and computational estimations (SG12)

- Coord. D. Rochman (CH)
- Journal article under review
- Review of experimental and simulation knowledge pertaining to DH assessments
- Incl. uncertainties, methods, standards.



## Decay heat comparison exercise (SG16)

- Coord. D. Rochman (CH)
- Kick off scheduled Sept 2024
- Pin-cell and assembly simulation with experimental data

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5. **NEA tools**

# Working Party on international nuclear data Evaluation Cooperation (WPEC)

Chair Dr Arjan PLOMPEN (EU),  
Secretariat Dr Andrew Holcomb



- ND
- AM
- IE

Working Party

**WPEC**

*US leadership highlighted*

Longer term Expert Groups

**EG HPRL**

**EG GNDS**  
*C. Mattoon (LLNL)*

Recently closed subgroups

**SG45 VaNDaL**  
*W. Haeck (LANL)*

**SG46 IE for ND evaluation**

**SG47 SINBAD**

**SG48 TSL**

**SG49 Reproducibility**  
*M. Herman (LANL)*

Currently running subgroups

**SG50 EXFOR**  
*A. Lewis (NNL), D. Neudecker (LANL)*

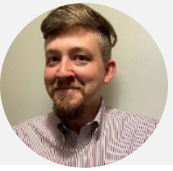
**SG51 URR**  
*V. Sobes (UTK), J. Brown (ORNL)*

Potential areas for new subgroups include gamma data, fission product yields, continuations of SG46 and SG48 work



# MCNP to Serpent-2

WPEC SG45 contributions from Steven van der Marck and Andrej Trkov



The screenshot shows the GitLab interface for the Serpent-2 repository. The top part displays a merge commit titled "Merge branch 'add-email' into 'main'" with a commit hash of 8f17e3a9. Below this, a file list shows various files and folders, including .gitignore, README.md, convert.sh, fix\_boundary.py, fix\_materials.py, and iaea\_map.py. The bottom part of the screenshot shows a table of example results for the full set of working models with reference calculation results using ENDF-7.1 data with Serpent-2.2.0.

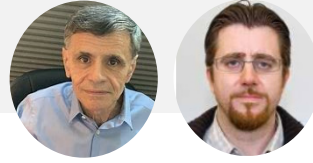
input	k-eff	uncertainty
iaea/heu/met/fast/001/hmf001.i.serp	0.99954	0.00029
iaea/heu/met/fast/001/hmf001_INL.i.serp	1.00007	0.00028
iaea/heu/met/fast/002/hmf002-001.i.serp	1.00163	0.0003
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- ~2400 functional inputs, ~4500 inputs with at least a starting point
- Upon request through the NEA Data Bank
- **Incentive to include in ICSBEP DVD**

- 
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(input from O. Buss)
  5. **NEA tools**

# Working Party on scientific issues and uncertainty analysis of Reactor Systems (WPRS)

Chairs K. Ivanov (US NCSU) & H. Ferroukhi (CH),  
Secretariat O. Buss



WPRS

 US leadership highlighted

IPD

  
**EG Physics of Reactor Systems (EGPRS)**

*H. Ferroukhi (CH), R. Grove (US ORNL)*

  
**EG Reactor System Multi Physics (EGMUP)**

*T. Valentine (US), E. Ivanov (FR)*

**International Reactor Physics Experiments Technical Review Group (IRPhE) \*\***

*M. DeHart (US INL), L. Buiron (FR)* 

  
**EG Reactor Core Thermal Hydraulics and Mechanics (EGTHM)**

*M. Avramova (US NCSU), A. Petruzzi (IT)*

**EG Reactor Fuel Performance (EGRFP)\***

**Task force Zero Power Reactors (ZPR)**

IE

\* Secretariat M. Bales  
\*\* Secretariat I. Hill

# Task Force on Zero Power Reactors “The demise of ZPRs – From concern to action” [2022-...]

IE

Chair: Robert Jacqmin (FR)

## – Objectives

- Assess the projected needs for new reactor physics validation data, and elaborate on the motivations
- Recommend a consensual course of action for acquiring such data, including minimal functional specifications of the needed facilities and expertise

## – Deliverables

- NSC/WPRS Workshop on Zero Power Reactors “The demise of zero power reactors: From concern to action”
- “WPRS Zero Power Reactor Task Force - Interview Questionnaire” , including **representatives from the criticality safety community**

A dedicated activity under the WPNCs reported on “Experimental needs for criticality safety purposes” including needs for ZPR facilities (NEA/NSC/R(2022)6, Sept. 2023). These criticality-oriented needs will be further conveyed by this ZPR TF. [oe.cd/wpncsexpneeds]

### Tentative structure of report (January '23)

1. Motivation, scope and objectives
2. Method of work: questionnaire, interviews of ZPR data providers and users, workshops, brainstorming,
3. ZPRs: how they are/were used, purpose, contribution to the development of nuclear reactors, lessons learned from their operation, including difficulties they face(d)
4. How/why we got where we are
5. Needs and role/value of ZPRs. Projected needs for neutron physics data, esp. SOA simulations. Anticipated challenges
6. Conclusion. Recommendations on how to meet these needs with (new) ZPRs and possibly other facilities, required instrumentation and expertise. International cooperation and business model considerations + how to implement

# SINBAD - Shielding Integral Benchmark Archive and Database, a task force

IPD

Chair: T. Miller (US)

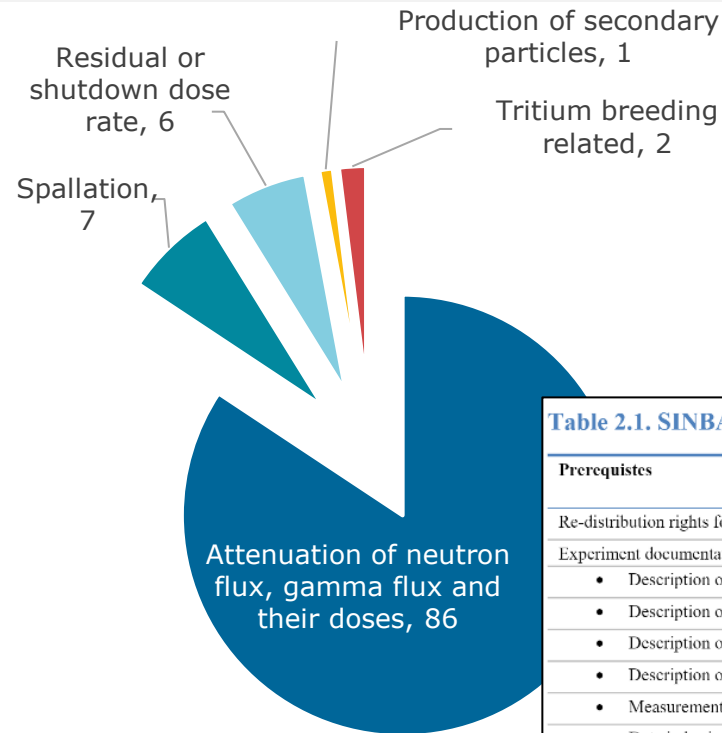


Table 2.1. SINBAD Maturity Levels

Prerequisites	Maturity Level				
	1	2	3	4	5
Re-distribution rights for experimental data and documentation has been obtained	<input checked="" type="checkbox"/>				
Experiment documentation according to Section 1 of Evaluation Guide (SINBAD TRG, 2019 <sup>[31]</sup> )					
• Description of the measurement facility		<input checked="" type="checkbox"/>			
• Description of each measurement configuration		<input checked="" type="checkbox"/>			
• Description of materials		<input checked="" type="checkbox"/>			
• Description of radiation sources		<input checked="" type="checkbox"/>			
• Measurement of input and output variables		<input checked="" type="checkbox"/>			
• Data in basic machine-readable format (e.g. CSV)		<input checked="" type="checkbox"/>			
Evaluation of measurement data (Section 2 of Evaluation Guide (SINBAD TRG, 2019 <sup>[31]</sup> ))					
• Evaluation of measurement configuration			<input checked="" type="checkbox"/>		
• Evaluation of radiation source			<input checked="" type="checkbox"/>		
• Evaluation of the measured data			<input checked="" type="checkbox"/>		
• Provision of computation models which have been used for evaluations			<input checked="" type="checkbox"/>		
• Sensitivity and uncertainty analysis (including provision of computation models)				<input checked="" type="checkbox"/>	
• Definition of scientific relevance based on 1*-3* scheme				<input checked="" type="checkbox"/>	
Benchmark model (Section 3 of Evaluation Guide (SINBAD TRG, 2019 <sup>[31]</sup> ))				<input checked="" type="checkbox"/>	
Sample case results and input files for related computational models (Section 4 of Evaluation Guide (SINBAD TRG, 2019 <sup>[31]</sup> ))				<input checked="" type="checkbox"/>	
Geometry as CAD model					<input checked="" type="checkbox"/>
Data in hierarchical machine readable data format (e.g. HDF5) <sup>1</sup>					<input checked="" type="checkbox"/>
Provision of automatic pre- & post-processing chain for the benchmark models					<input checked="" type="checkbox"/>

NEA & US RSICC, since 1992

Reactor (48), fusion neutronics (31), and accelerator (23) shielding experiments

TF: **Progressive**, well-defined **maintenance process**: subsequent updates to reach well-defined intermediate maturity levels

**Modular organization**: task force splits in several subgroups working on different entries/experiments

Q1 2024: Distributing SINBAD via NEA GitLab platform for SINBAD licensees

- Users can propose improvements to SINBAD via GitLab merge request workflows.
- Crowdsourcing, transparency and traceability

<https://oe.cd/nea-sinbad>

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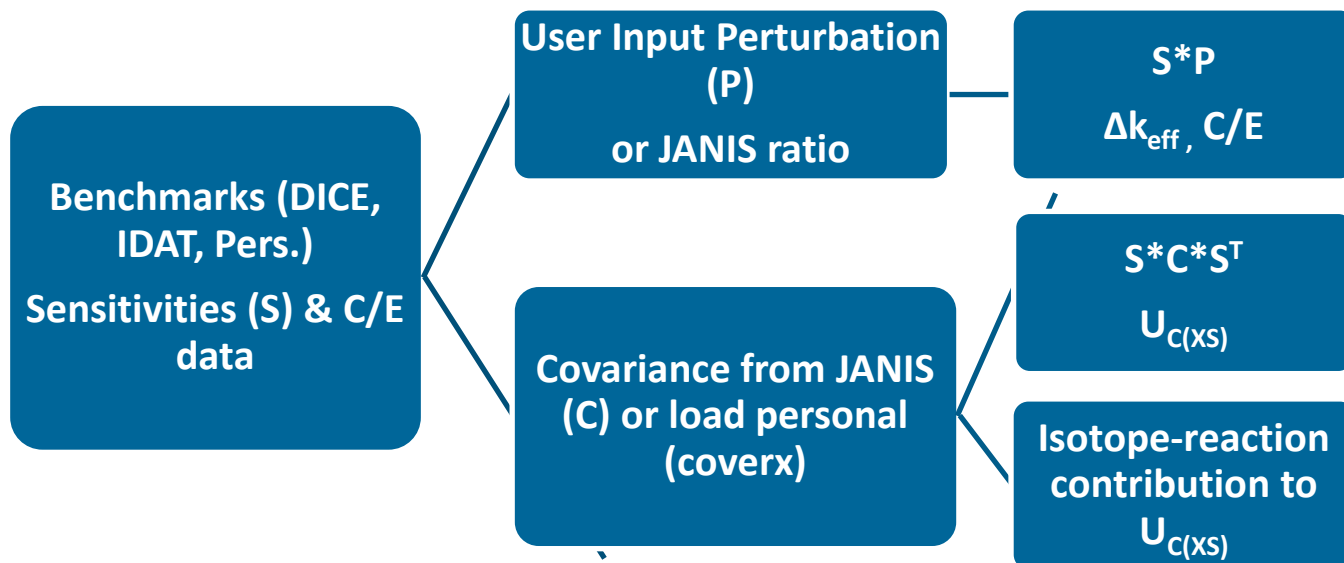
## 3. WPEC

## 4. WPRS

## 5. **NEA tools** (input from I. Hill)

*Nuclear Data Sensitivity Tool, Testing New Nuclear Data Libraries ENDF/B-VIII.1 (beta) and JEFF-4 Test Libraries, I. Hill & N. Soppera, in M&C 2023*

# NDaST Nuclear Data Sensitivity Tool



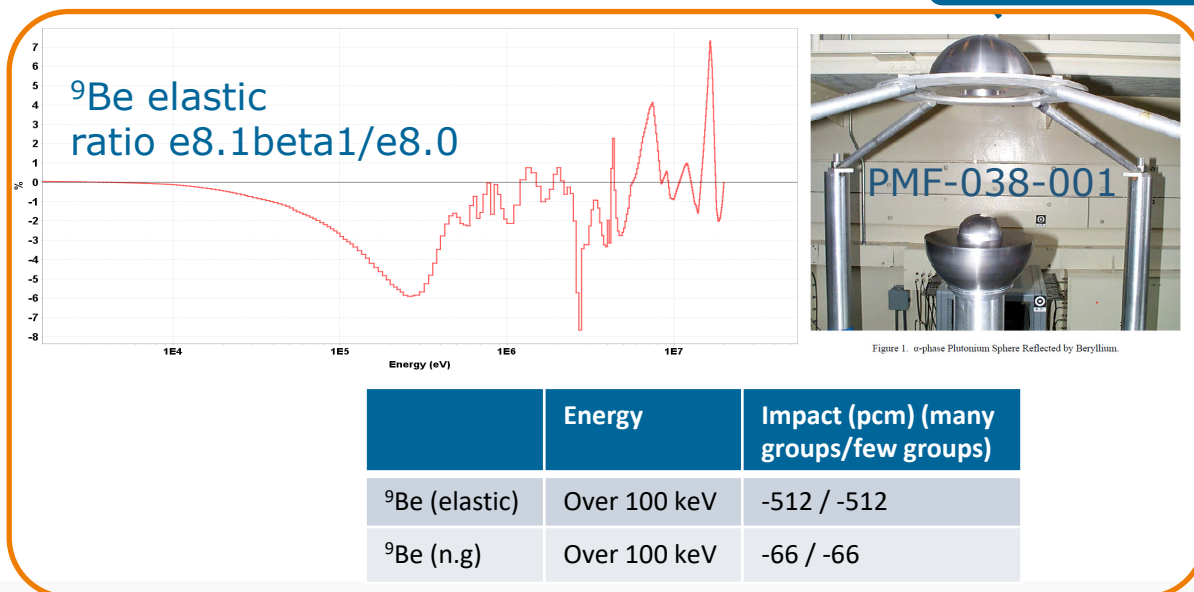
NDaST combines collections of sensitivity coefficients, with nuclear data tools.

Independent of response function, code, nuclear data library

Computations take of the order of **minutes**. Direct methods take multiple months to provide similar feedback.

Checked against direct perturbations in other work, found to be reliable in most cases, but accuracy is limited like other first order methods.

NDaST provides quick feedback to questions like, if I change from ENDF/B-VIII.0 to ENDF/B-VIII.1β1 how will my design calculations be impacted? Do the nuclear data changes improve the consistency of various benchmarks? Can filter what needs **additional checking**.





The NEA is grateful for the support of the NCSP for the US representation in NSC groups

Thank you for your attention

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